



Calculation and Measurement of the Moderator Temperature Coefficient of Reactivity for Pressurized Water Reactors

An American National Standard

Published by the
American Nuclear Society
555 N. Kensington Ave
La Grange Park, IL 60526

ANSI/ANS-19.11-2017



ANSI/ANS-19.11-2017

**American National Standard
Calculation and Measurement of the
Moderator Temperature Coefficient of Reactivity
for Pressurized Water Reactors**

Secretariat
American Nuclear Society

Prepared by the
**American Nuclear Society
Standards Committee
Working Group ANS-19.11**

Published by the
**American Nuclear Society
555 North Kensington Avenue
La Grange Park, Illinois 60526 USA**

Approved April 11, 2017
by the
American National Standards Institute, Inc.

American National Standard

Designation of this document as an American National Standard attests that the principles of openness and due process have been followed in the approval procedure and that a consensus of those directly and materially affected by the standard has been achieved.

This standard was developed under the procedures of the Standards Committee of the American Nuclear Society; these procedures are accredited by the American National Standards Institute, Inc., as meeting the criteria for American National Standards. The consensus committee that approved the standard was balanced to ensure that competent, concerned, and varied interests have had an opportunity to participate.

An American National Standard is intended to aid industry, consumers, governmental agencies, and general interest groups. Its use is entirely voluntary. The existence of an American National Standard, in and of itself, does not preclude anyone from manufacturing, marketing, purchasing, or using products, processes, or procedures not conforming to the standard.

By publication of this standard, the American Nuclear Society does not insure anyone utilizing the standard against liability allegedly arising from or after its use. The content of this standard reflects acceptable practice at the time of its approval and publication. Changes, if any, occurring through developments in the state of the art, may be considered at the time that the standard is subjected to periodic review. It may be reaffirmed, revised, or withdrawn at any time in accordance with established procedures. Users of this standard are cautioned to determine the validity of copies in their possession and to establish that they are of the latest issue.

The American Nuclear Society accepts no responsibility for interpretations of this standard made by any individual or by any ad hoc group of individuals. Inquiries about requirements, recommendations, and/or permissive statements (i.e., "shall," "should," and "may," respectively) should be sent to the Scientific Publications and Standards Department at Society Headquarters. Action will be taken to provide appropriate response in accordance with established procedures that ensure consensus.

Comments on this standard are encouraged and should be sent to Society Headquarters.

Published by

American Nuclear Society
555 North Kensington Avenue
La Grange Park, Illinois 60526 USA



This document is copyright protected.

Copyright © 2017 by American Nuclear Society. All rights reserved.

Any part of this standard may be quoted. Credit lines should read "Extracted from American National Standard ANSI/ANS-19.11-2017 with permission of the publisher, the American Nuclear Society." Reproduction prohibited under copyright convention unless written permission is granted by the American Nuclear Society.

Printed in the United States of America

Inquiry Requests

The American Nuclear Society (ANS) Standards Committee will provide responses to inquiries about requirements, recommendations, and/or permissive statements (i.e., “shall,” “should,” and “may,” respectively) in American National Standards that are developed and approved by ANS. Responses to inquiries will be provided according to the Policy Manual for the ANS Standards Committee. Nonrelevant inquiries or those concerning unrelated subjects will be returned with appropriate explanation. ANS does not develop case interpretations of requirements in a standard that are applicable to a specific design, operation, facility, or other unique situation only and therefore is not intended for generic application.

Responses to inquiries on standards are published in ANS’s magazine, *Nuclear News*, and are available publicly on the ANS website or by contacting the Scientific Publications and Standards Department.

Inquiry Format

Inquiry requests shall include the following:

- (1) the name, company name if applicable, mailing address, and telephone number of the inquirer;
- (2) reference to the applicable standard edition, section, paragraph, figure, and/or table;
- (3) the purpose(s) of the inquiry;
- (4) the inquiry stated in a clear, concise manner;
- (5) a proposed reply, if the inquirer is in a position to offer one.

Inquiries should be addressed to:

American Nuclear Society
Scientific Publications and Standards Department
555 N. Kensington Avenue
La Grange Park, IL 60526

or standards@ans.org

This is a preview of "ANSI/ANS-19.11-2017". [Click here to purchase the full version from the ANSI store.](#)

American National Standard ANSI/ANS-19.11-2017

Foreword (This foreword is not a part of American National Standard “Calculation and Measurement of the Moderator Temperature Coefficient of Reactivity for Pressurized Water Reactors,” ANSI/ANS-19.11-2017.)

It is the intent of this American National Standard to provide guidance and specify criteria for the calculation and measurement of the moderator temperature coefficient of reactivity (MTC) in pressurized water reactors (PWRs). The MTC is a major designed-in safety feature in PWRs. These reactors are designed to maintain a negative MTC over a large range of operating conditions. Although most off-nominal conditions benefit from a large negative MTC, some cooldown accidents are aggravated by the temperature feedback. For this reason it is important to determine the MTC accurately. This standard provides guidance and specifies criteria for determining the MTC in a PWR. Measurement of the isothermal temperature coefficient of reactivity (ITC) at hot-zero-power (HZP) conditions is covered in ANSI/ANS-19.6.1-2011 (R2016), “Reload Startup Physics Tests for Pressurized Water Reactors.” The current standard therefore addresses the calculation of the ITC at HZP and the calculation and measurement of the MTC at power.

Major differences between the current edition and the earlier edition of this standard are the following:

- The basis/reason for adding the section regarding the use of precalculated coefficients for the boron exchange test method are discussed;
- Editorial changes were made to reflect a consistent format throughout the standard, and equations were renumbered to accommodate the addition of a data reduction technique using precalculated coefficients in the measurement of MTC using the boron exchange test method;
- The term “fuel assembly” is not used in the standard and was removed from the definitions of terms in Sec. 3.4. The term “full power” was replaced with “hot full power” as its acronym “HFP” is used throughout the standard;
- The statepoint equations used in the test simulation method for calculating the correction terms used in data reduction of the boron exchange method were revised because they erroneously contained the measured total temperature coefficient of reactivity and were double counted for the reactivity change;
- The “Advantages” and “Disadvantages” associated with each test method in the standard are illustrative in nature and do not represent requirements and were moved to the Appendices to facilitate clarity regarding the technical detail and requirements for each test method. The advantages and disadvantages deal primarily with cost and time associated with each method that the user may wish to consider in selecting one test method over another.

This standard might reference documents and other standards that have been superseded or withdrawn at the time the standard is applied. A statement has been included in the reference section that provides guidance on the use of references.

This standard does not incorporate the concepts of generating risk-informed insights, performance-based requirements, or a graded approach to quality assurance. The user is advised that one or more of these techniques could enhance the application of this standard.

This standard was developed by the ANS-19.11 Working Group of the American Nuclear Society. During the period the standard was revised, the working group had the active participation of the following members:

M. Mahgerefteh (Chair), *Exelon Generation LLC*

S. P. Baker, *Transware Enterprises*
R. J. Borland, *FirstEnergy Nuclear Operating Company*
D. Brown, *Tennessee Valley Authority*
M. Eckenrode, *AREVA Inc.*
E. Knuckles, *Individual*

The membership of the Reactor Physics Subcommittee (ANS-19) at the time of its review and approval of this standard was the following:

D. M. Cokinos (Chair), *Brookhaven National Laboratory*
C. T. Rombough (Secretary), *CTR Technical Services, Inc.*

A. Attard, *U.S. Nuclear Regulatory Commission*
S. P. Baker, *Transware Enterprises*
J. Bess, *Idaho National Laboratory*
M. C. Brady Raap, *Individual*
R. T. Chiang, *Individual*
M. DeHart, *Idaho National Laboratory*
D. J. Diamond, *Brookhaven National Laboratory*
M. Eckenrode, *AREVA Inc.*
I. Gauld, *Oak Ridge National Laboratory*
A. Haghghat, *Virginia Tech Research Center*
J. Katakura, *Japan Atomic Energy Agency*
E. Knuckles, *Individual*
R. Little, *Los Alamos National Laboratory*
M. Mahgerefteh, *Exelon Generation LLC*
E. Nichita, *University of Ontario Institute of Technology*
B. Rouben, *Individual*
A. Weitzberg, *Individual*
W. B. Wilson, *Individual*

The Safety and Radiological Analysis Consensus Committee had the following membership at the time it reviewed and approved this standard:

A. O. Smetana (Chair), *Savannah River National Laboratory*
A. Weitzberg (Vice Chair), *Individual*
F. A. Alpan, *Westinghouse Electric Company, LLC*
R. Amato, *Individual*
M. C. Brady Raap, *Individual*
D. M. Cokinos, *Brookhaven National Laboratory*
D. Dudziak, *Los Alamos National Laboratory*
C. Graham, *Health Physics Society Representative (Employed by Ameren)*
J. Jarvis, *Bechtel Corporation*
M. Gupta, *AECOM-Professional Solutions*
M. Hertel, *Georgia Institute of Technology*
P. Hulse, *Sellafield Ltd.*
D. Palmrose, *U.S. Nuclear Regulatory Commission*
C. T. Rombough, *CTR Technical Services, Inc.*
C. Sanders, *University of Nevada, Las Vegas*

Contents

Section	Page
1 Introduction	1
2 Scope	2
3 Acronyms and definitions.....	2
3.1 Acronyms	2
3.2 Shall, should, and may	3
3.3 Limitations.....	3
3.4 Definitions	3
4 Moderator temperature coefficient and reactivity	5
4.1 Moderator temperature coefficient	5
4.2 Moderator temperature coefficient of reactivity	5
5 Calculation of the MTC.....	5
5.1 Numerics and convergence.....	6
5.1.1 Magnitude of the moderator temperature change.....	6
5.1.2 Finite-difference representation for the MTC	6
5.1.3 Convergence criterion for k_{eff}	7
5.2 Zero power calculation.....	7
5.3 At-power calculation	8
6 Measurement of the MTC at power.....	8
6.1 Measurement techniques	9
6.1.1 Common requirements	9
6.1.2 Boron exchange	11
6.1.3 Control rod exchange with measured control rod worth	16
6.1.4 Control rod exchange with calculated control rod worth	17
6.1.5 Power exchange.....	19
6.1.6 Depletion	21
6.2 Extrapolation.....	21
7 Documentation.....	22
8 Reference.....	22
Appendices	
Appendix A Advantages and Disadvantages of Test Methods	23
Appendix B Advantages and Disadvantages of Boron Exchange Data Reduction Methods.....	26

This is a preview of "ANSI/ANS-19.11-2017". [Click here to purchase the full version from the ANSI store.](#)

American National Standard ANSI/ANS-19.11-2017

Calculation and Measurement of the Moderator Temperature Coefficient of Reactivity for Pressurized Water Reactors

1 Introduction

The moderator temperature coefficient of reactivity (MTC) relates a change in core reactivity to a change in reactor moderator temperature: A positive MTC means that reactivity increases with increasing moderator temperature while a negative MTC means that reactivity decreases with increasing moderator temperature. The MTC is a major designed-in safety feature in pressurized water reactors (PWRs). These reactors are designed to maintain a negative MTC over a large range of operating conditions. Therefore, an increase in the moderator temperature will cause the reactivity to decrease, so that the moderator temperature tends to return to its initial value. Reactivity increases that cause an increase in moderator temperature thus will be self-limiting, and stable power operation will result. Although most off-nominal conditions benefit from a large negative MTC, some cooldown accidents are aggravated by the temperature feedback. For this reason it is important to determine the MTC accurately.

The value of the MTC is determined by competing effects. The decrease in the density of water that accompanies an increase in temperature leads to a reduction in neutron moderation, which tends to make the MTC more negative. The MTC is made more positive, however, by the addition of soluble boron to the water. Boron is a strong neutron absorber, and because the density of the boron is directly proportional to the density of the water, the absorption rate due to boron decreases as the water density decreases.

A change in moderator temperature affects core reactivity both directly and indirectly. It affects core reactivity directly at a macroscopic level through the change in moderator density that accompanies a change in the moderator temperature, as discussed above. It also affects neutron moderation directly at a microscopic level because the neutron scattering kernel for the water molecule is weakly temperature dependent. Furthermore, these changes in moderation have secondary effects, such as redistribution of the flux shape.

For the purposes of this standard, the MTC is defined to include all such effects, whether direct or indirect:

$$\text{MTC} \equiv \left(\frac{\partial \rho}{\partial T_{ave}} \right)_{\mu} + \left(\frac{\partial \rho}{\partial \mu_{ave}} \right)_{p} \times \left(\frac{\partial \mu_{ave}}{\partial T_{ave}} \right)_{p} + \left(\frac{\partial \rho}{\partial \phi} \right)_{p} \times \left(\frac{\partial \phi}{\partial T_{ave}} \right)_{p},$$

(Eq. 1)

where:

ρ is the reactivity;