

# American Nuclear Society

## evaluation of subsurface radionuclide transport at commercial nuclear power plants

### an American National Standard

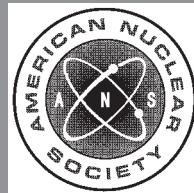
**REAFFIRMED**

**March 10, 2016**

**ANSI/ANS-2.17-2010; R2016**

This standard has been reviewed and reaffirmed with the recognition that it may reference other standards and documents that may have been superseded or withdrawn. The requirements of this document will be met by using the version of the standards and documents referenced herein. It is the responsibility of the user to review each of the references and to determine whether the use of the original references or more recent versions is appropriate for the facility. Variations from the standards and documents referenced in this standard should be evaluated and documented.

This standard does not necessarily reflect recent industry initiatives for risk informed decision-making or a graded approach to quality assurance. Users should consider the use of these industry initiatives in the application of this standard.



published by the  
American Nuclear Society  
555 North Kensington Avenue  
La Grange Park, Illinois 60526 USA

ANSI/ANS-2.17-2010

**American National Standard  
Evaluation of Subsurface  
Radionuclide Transport at Commercial  
Nuclear Power Plants**

Secretariat  
**American Nuclear Society**

Prepared by the  
**American Nuclear Society  
Standards Committee  
Working Group ANS-2.17**

Published by the  
**American Nuclear Society  
555 North Kensington Avenue  
La Grange Park, Illinois 60526 USA**

Approved December 23, 2010  
by the  
**American National Standards Institute, Inc.**

## **American National Standard**

Designation of this document as an American National Standard attests that the principles of openness and due process have been followed in the approval procedure and that a consensus of those directly and materially affected by the standard has been achieved.

This standard was developed under procedures of the Standards Committee of the American Nuclear Society; these procedures are accredited by the American National Standards Institute, Inc., as meeting the criteria for American National Standards. The consensus committee that approved the standard was balanced to ensure that competent, concerned, and varied interests have had an opportunity to participate.

An American National Standard is intended to aid industry, consumers, governmental agencies, and general interest groups. Its use is entirely voluntary. The existence of an American National Standard, in and of itself, does not preclude anyone from manufacturing, marketing, purchasing, or using products, processes, or procedures not conforming to the standard.

By publication of this standard, the American Nuclear Society does not insure anyone utilizing the standard against liability allegedly arising from or after its use. The content of this standard reflects acceptable practice at the time of its approval and publication. Changes, if any, occurring through developments in the state of the art, may be considered at the time that the standard is subjected to periodic review. It may be reaffirmed, revised, or withdrawn at any time in accordance with established procedures. Users of this standard are cautioned to determine the validity of copies in their possession and to establish that they are of the latest issue.

The American Nuclear Society accepts no responsibility for interpretations of this standard made by any individual or by any ad hoc group of individuals. Requests for interpretation should be sent to the Standards Department at Society Headquarters. Action will be taken to provide appropriate response in accordance with established procedures that ensure consensus on the interpretation.

Comments on this standard are encouraged and should be sent to Society Headquarters.

Published by

**American Nuclear Society  
555 North Kensington Avenue  
La Grange Park, Illinois 60526 USA**

Copyright © 2010 by American Nuclear Society. All rights reserved.

Any part of this standard may be quoted. Credit lines should read "Extracted from American National Standard ANSI/ANS-2.17-2010 with permission of the publisher, the American Nuclear Society." Reproduction prohibited under copyright convention unless written permission is granted by the American Nuclear Society.

Printed in the United States of America

**Foreword** (This Foreword is not a part of American National Standard "Evaluation of Subsurface Radionuclide Transport at Commercial Nuclear Power Plants," ANSI/ANS-2.17-2010.)

This standard constitutes a major revision of the original standard, ANSI/ANS-2.17-1980, which was adopted on April 9, 1980, reaffirmed on October 3, 1989, and withdrawn on July 28, 2000. A new working group, Working Group ANS-2.17 of ANS-25 Subcommittee (Siting: Environmental & Emergency Preparedness) of the American Nuclear Standards Committee, was constituted November 2005 to revise the original standard.

This standard might reference documents and other standards that have been superseded or withdrawn at the time the standard is applied. A statement has been included in the references section that provides guidance on the use of references.

This standard mentions, but does not exhaustively describe, the concepts of generating risk-informed insights, performance-based requirements, and a graded approach to quality assurance. The user is advised that one or more of these techniques could enhance the application of this standard.

Two appendices are provided to assist practitioners who would implement the guidance in this standard. Appendix A provides information on relevant U.S. Nuclear Regulatory Commission regulatory criteria and guidance, and its Table A provides a listing of standard documents (e.g., ANS, ASTM, ISO, etc.) for conducting subsurface radionuclide transport characterization, monitoring, and modeling programs. Appendix B provides tables that summarize information and parameters identified in the guidance.

The ANS-2.17 Working Group of the Standards Committee of the American Nuclear Society had the following membership:

J. S. Bollinger (Cochair), *Savannah River National Laboratory*  
T. C. Rasmussen (Cochair), *University of Georgia*

M. J. Barvenik, *GZA GeoEnvironmental, Inc.*  
R. L. Beauheim, *Sandia National Laboratories*  
G. S. Bodvarsson, *Lawrence Berkeley National Laboratory*<sup>1)</sup>  
J. M. Godfrey, *Southern Nuclear Company, Alabama Power Company*  
D. Goswami, *Washington State Department of Ecology*  
V. Guvanasekaran, *HydroGeoLogic, Inc.*  
C. D. Martinec, *Duke Energy*  
P. D. Meyer, *Pacific Northwest National Laboratory*  
F. J. Molz III, *Clemson University*  
T. J. Nicholson, *U.S. Nuclear Regulatory Commission*  
D. Scott, *Radiation Safety and Control Services, Inc.*  
E. P. Weeks, *U.S. Geological Survey*  
D. G. Wells, *Washington Savannah River Company*  
M. H. Young, *Desert Research Institute*

This standard was prepared under the guidance of Subcommittee ANS-25, Siting: Environmental & Emergency Preparedness, of the American Nuclear Society. At the time of the ballot, Subcommittee ANS-25 was composed of the following members:

K. R. Bryson (Chair), *Shaw Environmental, Inc.*  
C. A. Mazzola (Vice Chair), *Shaw Environmental & Infrastructure, Inc.*

J. S. Bollinger, *Savannah River National Laboratory*  
C. Costantino, *Individual*

---

<sup>1)</sup> The working group would like to gratefully acknowledge the contributions by G. S. "Bo" Bodvarsson, who died prior to the completion of this standard.

A. N. Findikakis, *Bechtel Corporation*  
C. Guggino, *Bechtel Corporation*  
D. Hang, *Individual*  
K. Hanson, *AMEC Geomatrix*  
J. J. Litehiser, *Bechtel Corporation*  
T. C. Rasmussen, *University of Georgia*  
J. D. Stevenson, *Individual*  
L. W. Vail, *Pacific Northwest National Laboratory*  
S. A. Vigeant, *Shaw Environmental & Infrastructure*

The standard was processed and approved for submittal to ANSI by the Nuclear Facilities Standards Committee (NFSC) of the American Nuclear Society. At the time it approved this standard, the NFSC had the following membership:

C. A. Mazzola (Chair), *Shaw Environmental & Infrastructure, Inc.*  
R. M. Ruby (Vice Chair), *Constellation Energy*

J. K. August, *CORE, Inc.*  
W. H. Bell, *South Carolina Electric & Gas Company*  
J. R. Brault, *Shaw MOX Project*  
C. K. Brown, *Southern Nuclear Operating Company*  
R. H. Bryan, *Tennessee Valley Authority*  
K. R. Bryson, *Shaw Environmental, Inc.*  
C. E. Carpenter, *U.S. Nuclear Regulatory Commission*  
D. R. Eggett, *Automated Engineering Services Corporation*  
R. W. Englehart, *Individual*  
P. Guha, *U.S. Department of Energy*  
R. Hall, *Exelon Generation Company, LLC*  
P. S. Hastings, *Duke Energy*  
R. A. Hill, *ERIN Engineering and Research, Inc.*  
N. P. Kadambi, *Individual*  
E. M. Lloyd, *Exitech Corporation*  
S. A. Lott, *Los Alamos National Laboratory*  
J. E. Love, *Bechtel Corporation*  
R. H. McFetridge, *Westinghouse Electric Corporation*  
C. H. Moseley, *ASME/NQA Liaison (BWXT Y-12)*  
D. G. Newton, *AREVA NP*  
W. N. Prillaman, *AREVA NP*  
W. B. Reuland, *Individual*  
D. J. Spellman, *Oak Ridge National Laboratory*  
S. L. Stamm, *Shaw Nuclear Services*  
J. D. Stevenson, *Individual*  
J. A. Wehrenberg, *Southern Nuclear Operating Company*  
M. J. Wright, *Entergy Operations, Inc.*

<b>Contents</b>	<b>Section</b>	<b>Page</b>
	<b>1</b> Scope .....	1
	<b>2</b> Definitions .....	1
	2.1 Acronyms and initialisms .....	1
	2.2 Definition of terms .....	1
	<b>3</b> Performance assessment methodology .....	5
	3.1 Performance assessment activities .....	5
	3.2 Performance assessment elements .....	6
	<b>4</b> Site characterization .....	8
	4.1 Conceptual site model .....	8
	4.2 Facilities characterization .....	9
	4.3 Hydrogeologic characterization .....	10
	<b>5</b> Mathematical modeling .....	13
	5.1 Model scope .....	13
	5.2 Calibration, prediction, and updating .....	16
	5.3 Uncertainty assessment .....	17
	<b>6</b> Performance-confirmation monitoring .....	17
	6.1 Types of monitoring data .....	17
	6.2 Monitoring methods .....	18
	6.3 Monitoring locations and frequencies .....	18
	<b>7</b> Information management .....	20
	7.1 Characterization and monitoring data management .....	20
	7.2 Mathematical model management .....	21
	<b>8</b> References .....	21
<b>Appendices</b>		
	Appendix A Consultation and Coordination .....	25
	Appendix B Summary of Information and Parameters .....	31
<b>Table</b>		
	Table 1	7
	Ranked list of radionuclides at commercial nuclear power plants (typical pressurized water reactor) based on their relative abundance, activity, and transport characteristics .....	
<b>Figure</b>		
	Figure 1	6
	Flowchart describing performance assessment activities and the relationships among these activities .....	

This is a preview of "ANSI/ANS-2.17-2010 (...". [Click here](#) to purchase the full version from the ANSI store.

# Evaluation of Subsurface Radionuclide Transport at Commercial Nuclear Power Plants

## 1 Scope

This national standard establishes the requirements for evaluating the occurrence and movement of radionuclides in the subsurface resulting from abnormal radionuclide releases at commercial nuclear power plants.

This standard applies to abnormal radionuclide releases that affect groundwater, water supplies derived from groundwater, and surface waters affected by subsurface transport, including exposure pathways across the groundwater-surface-water transition zone.

This standard does not apply to the following:

- subsurface occurrence and movement of non-radioactive materials, other than as indicators of subsurface radionuclide occurrence and movement in soil and groundwater;
- surface occurrence and movement of radionuclides, except to the extent that surface radionuclide occurrence and movement might affect, or be affected by, on-site subsurface radionuclide occurrence and movement (e.g., a surface release that subsequently infiltrates and affects groundwater, a subsurface release that affects surface water, including exposure pathways across the groundwater-surface-water transition zone);
- corrective action, which might be required as the result of a subsurface radionuclide release;
- dose calculations to demonstrate compliance with any regulatory requirement.

## 2 Definitions

### 2.1 Acronyms and initialisms

**ALARA:** as low as is reasonably achievable

**ANS:** American Nuclear Society

**ANSI:** American National Standards Institute

**ASTM:** ASTM International, previously known as the American Society for Testing and Materials

**CFR:** *Code of Federal Regulations*

**CRWMS:** civilian radioactive waste management system management and operating contractor

**CSM:** conceptual site model

**DQO(s):** data quality objective(s)

**EIS:** Environmental Impact Statement

**EPRI:** Electric Power Research Institute

**FEP(s):** feature(s), event(s), and process(es)

**IAEA:** International Atomic Energy Agency

**NEI:** Nuclear Energy Institute

**NGWA:** National Ground Water Association

**NRC:** National Research Council

**REMP:** Radiological Environmental Monitoring Program

**RETS:** Radioactive Effluent Technical Specifications

**TEDE:** total effective dose equivalent

**USEPA:** U.S. Environmental Protection Agency

**USNRC:** U.S. Nuclear Regulatory Commission

### 2.2 Definition of terms

**abnormal radionuclide release:** The unplanned or uncontrolled emission of an effluent (i.e., containing plant-related, licensed radioactive material).

**as low as is reasonably achievable (ALARA):** Every reasonable effort is made to maintain exposures to radiation as far below the dose limits as is practical consistent with the purpose for which the licensed activity is undertaken, taking into account the state of technology, the economics of improvements in relation to state of technology, the economics of improvements in relation to benefits to the public health and