

American Nuclear Society

REAFFIRMED

March 20, 2002

ANSI/ANS-8.12-1987

(R2002)

**nuclear criticality control and safety of
plutonium-uranium fuel mixtures outside reactors**

an American National Standard

REAFFIRMED

February 11, 2011

ANSI/ANS-8.12-1987

(R2011)

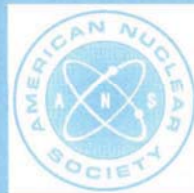
May 6, 2016

ANSI/ANS-8.12-1987

(R2016)

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published by the

American Nuclear Society

555 North Kensington Avenue

La Grange Park, Illinois 60526 USA

ANSI/ANS-8.12-1987
Revision of
ANSI/ANS-8.12-1978

**American National Standard
for Nuclear Criticality Control and Safety of
Plutonium-Uranium Fuel Mixtures Outside Reactors**

Secretariat
American Nuclear Society

Prepared by the
American Nuclear Society
Standards Committee
Working Group ANS-8.12

Published by the
American Nuclear Society
555 North Kensington Avenue
La Grange Park, Illinois 60525 USA

Approved September 11, 1987
by the
American National Standards Institute, Inc.

American National Standard

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Printed in the United States of America

Foreword (This Foreword is not a part of American National Standard for Nuclear Criticality Control and Safety of Plutonium-Uranium Fuel Mixtures Outside Reactors, ANSI/ANS-8.12-1987.)

This standard provides guidance for the prevention of criticality accidents in the handling, storing, processing, and transporting of plutonium-uranium fuel mixtures outside reactors and is applicable to all operations involving mixtures of plutonium and natural uranium. It constitutes an extension of the American National Standard for Nuclear Criticality Safety in Operations with Fissionable Materials Outside Reactors, ANSI/ANS-8.1-1983.

Under the prescribed five year review of ANSI/ANS-8.12-1978, the standard has been revised to include subcritical limits for heterogeneous lattices of mixed oxide fuel pins in water. The basis for the limits for both homogeneous mixtures and for lattices are calculations done by several members of the work group, which have been published in the open literature. These calculations were done by methods that have been validated by correlations with available experimental data, and an adequate margin of subcriticality was allowed. The revised standard was prepared by Work Group ANS-8.12.1 of Subcommittee 8 of the Standards Committee of the American Nuclear Society. This work group was composed of:

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Nuclear Criticality Control and Safety of Plutonium-Uranium Fuel Mixtures Outside Reactors

1. Introduction

The American National Standard for Nuclear Criticality Safety in Operations with Fissionable Materials Outside Reactors, ANSI/ANS-8.1-1983 [1]¹ provides single parameter limits for fissionable units of simple shape containing ^{233}U , ^{235}U , and ^{239}Pu [1]. As an example of multi-parameter control it provides an increase in the limits of ^{235}U resulting from control of the amount of ^{238}U associated with ^{235}U in uranium metal and oxide enriched to no more than 5 wt% ^{235}U . Larger limits for plutonium likewise result from placing reliance on, and hence controlling, the isotopic concentration of ^{240}Pu and the amount of uranium associated with the plutonium. Such increases may prove valuable for operations with mixed oxides of plutonium and uranium encountered in light water, liquid metal fast breeder, and gas-cooled fast reactor fuel cycle operations.

The limits provided here were calculated (E. D. Clayton [2, 3]) by methods satisfying the requirements for validation of a calculational method as set forth in 4.3 of ANSI/ANS-8.1-1983 [1].

The administrative and technical practices for criticality safety and control as embodied in ANSI/ANS-8.1-1983 [1] and in American National Standard Administrative Practices for Nuclear Criticality Safety, ANSI/ANS-8.19-1984 [4] are applicable herein.

2. Scope

This standard is applicable to operations with plutonium-uranium oxide fuel mixtures outside nuclear reactors, except the assembly of these materials under controlled conditions, such as in critical experiments. Basic criteria are presented for plutonium-uranium fuel mixtures in single

¹Numbers in brackets refer to corresponding numbers in Section 7, References.

units of simple shape containing no more than 30 wt% plutonium combined with uranium containing no more than 0.71 wt% ^{235}U . The limits for uniform aqueous mixtures (solution) are applicable to homogeneous mixtures and slurries in which the particles constituting the mixture are uniformly distributed and have a diameter no larger than 127 μm (0.005 in.), i.e., are capable of being passed through 120 mesh screen.²

This standard does not include the details of administrative controls, the design of processes or equipment, the description of instrumentation for process control or detailed criteria to be met in transporting fissionable materials.

3. Definitions

3.1 Limitations. The definitions given below are of a restricted nature for the purpose of this standard. Other specialized terms are defined in H. Alter, et al., "Glossary of Terms in Nuclear Science and Technology" [5].

3.2 Shall, Should, and May. The word "shall" is used to denote a requirement, the word "should" to denote a recommendation, and the word "may" to denote permission, neither a requirement nor a recommendation. In order to conform with this standard all operations shall be performed in accordance with its requirements but not necessarily with its recommendations.

²Mixtures containing particles 60-100 μm in size have been considered by experimenters to be homogeneous [V. I. NEELEY and H. E. HANDLER, "Measurement of Multiplication Constant for Slightly Enriched Homogeneous UO_3 -Water Mixtures and Minimum Enrichment for Criticality," HW-70310 Hanford Atomic Products Operations (August 1961) and V. I. NEELEY, J. A. BERBERET and R. H. MASTERSON, " k_∞ of Three Weight Per Cent ^{235}U Enriched UO_3 and UO_2 (NO_3)₂ Hydrogenous Systems," HW-66882, Hanford Atomic Products Operations (September 1961)].