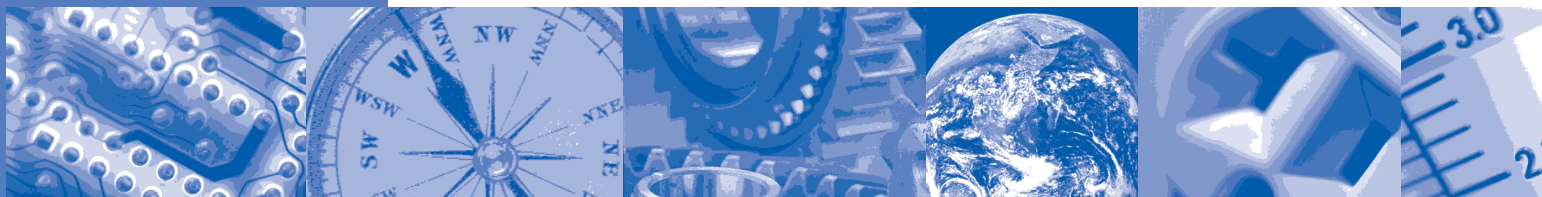


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Methodologies for the Determination of Setpoints for Nuclear Safety-Related Instrumentation



Approved 1 January 2000

ISA-RP67.04.02-2000

Methodologies for the Determination of Setpoints for Nuclear Safety-Related Instrumentation

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Preface

This preface is included for informational purposes and is not part of ISA-RP67.04.02.

This recommended practice has been prepared as part of the service of ISA, the international society for measurement and control, toward a goal of uniformity in the field of instrumentation. To be of real value, this document should not be static but should be subject to periodic review. Toward this end, the Society welcomes all comments and criticisms and asks that they be addressed to the Secretary, Standards and Practices Board; ISA; 67 Alexander Drive; P. O. Box 12277; Research Triangle Park, NC 27709; Telephone (919) 549-8411; Fax (919) 549-8288; E-mail: standards@isa.org.

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The ISA Standards and Practices Department is aware of the growing need for attention to the metric system of units in general and the International System of Units (SI) in particular, in the preparation of instrumentation standards, recommended practices, and technical reports. However, since this recommended practice does not provide constants or dimensional values for use in the manufacture or installation of equipment, English units are used in the examples provided.

Before utilizing this recommended practice, it is important that the user understand the relevance of instrument channel uncertainty and safety-related setpoint determination for nuclear power plants. Safety-related instrument setpoints are chosen so that potentially unsafe or damaging process excursions (transients) can be avoided and/or terminated prior to exceeding safety limits (process-design limits). The selection of a setpoint requires that consideration be given to much more than just instrumentation.

Experience has shown that an operational limit should be placed on critical process parameters to ensure that, given the most severe operating or accident transient, the plant's design safety limits will not be exceeded. Performance of an accident analysis establishes the analytical limits for critical process parameters. Typically, the accident analysis models include the thermodynamic, hydraulic, and mechanical dynamic response of the processes as well as assumptions regarding the time response of instrumentation. The analytical limits, as established by an accident analysis, do not normally include considerations for the accuracy (uncertainty) of installed instrumentation. To ensure that the actual trip setpoint of an instrument channel is appropriate, additional analysis may be necessary.

Instrument channel uncertainty should be determined, based on the characteristics of installed instrumentation, the environmental conditions present at the plant locations associated with the instrumentation, and on process conditions. A properly calculated setpoint will initiate a plant protective action before the process parameter exceeds its analytical limit, which, in turn, ensures that the transient will be avoided and/or terminated before the process parameter exceeds the established safety limit.

ISA-S67.04 was initially developed in the middle 1970s by the industry in response to large numbers of licensee event reports (LER). These LERs were attributed to the lack of adequate consideration of equipment drift characteristics when establishing the trip setpoints for the limiting safety system settings (LSSS) and engineered safety features actuation system (ESFAS) setpoints. These setpoints are included as part of a nuclear power plant's operating license in their technical specifications. Hence, bistable trip setpoints were found beyond the allowable values identified in the technical specifications.

The scope of the standard was focused on LSSS and ESFAS setpoints. As the standard evolved, it continued to focus on those key safety-related setpoints noted previously. It may also be noted that as the technical specifications have evolved, the values now included in the technical specifications may be the

trip setpoint or the allowable value or both depending on the setpoint methodology philosophy used by the plant and/or the Nuclear Steam Supply Systems (NSSS) vendor. The methodologies, assumptions, and conservatism associated with performing accident analyses and setpoint determinations, like other nuclear power plant technologies, have also evolved. This evolution has resulted in the present preference for explicit evaluation of instrument channel uncertainties and resulting setpoints rather than implicitly incorporating such uncertainties into the overall safety analyses. Both the explicit and implicit approaches can achieve the same objective of assuring that design safety limits will not be exceeded. During the process of developing the 1988 revision of ISA-S67.04, it was determined that, because of the evolving expectations concerning setpoint documentation, additional guidance was needed concerning methods for implementing the requirements of the standard. In order to address this need, standard Committees SP67.15 and SP67.04 were formed and prepared ISA-RP67.04, Part II, 1994. It is the intent of the Committees that the scope of the recommended practice be consistent with the scope of the standard. The recommended practice is to be utilized in conjunction with the standard. The standard is ISA-S67.04.01, and the recommended practice is ISA-RP67.04.02.

During the development of this recommended practice, a level of expectation for setpoint calculations has been identified, which, in the absence of any information on application to less critical setpoints, leads some users to come to expect that all setpoint calculations will contain the same level of rigor and detail. The lack of specific treatment of less critical setpoints has resulted in some potential users expecting the same detailed explicit consideration of all the uncertainty factors described in the recommended practice for all setpoints. It is not the intent of the recommended practice to suggest that the methodology described is applicable to all setpoints. Although it may be used for most setpoint calculations, it is by no means necessary that it may be used for all setpoints. In fact, in some cases, it may not be appropriate.

Setpoints associated with the analytical limits determined from the accident analyses are considered part of the plant's safety-related design since they are critical to ensuring the integrity of the multiple barriers to the release of fission products. This class of setpoints and their determination have historically been the focus of ISA-S67.04.01 as discussed above.

Also treated as part of many plants' safety-related designs are setpoints that are not determined from the accident analyses and are not required to maintain the integrity of the fission product barriers. These setpoints may provide anticipatory inputs to, or reside in, the reactor protection or engineered safeguards initiation functions but are not credited in any accident analysis. Alternatively, there are setpoints that support operation of, not initiation of, the engineered safety features.

In applying the standard to the determination of setpoints, a graduated or "graded" approach may be appropriate for setpoints that are not credited in the accident analyses to initiate reactor shutdown or the engineered safety features.

While it is the intent that the recommended practice will provide a basis for consistency in approach and terminology to the determination of setpoint uncertainty, it is acknowledged that the recommended practice is not an all-inclusive document. Other standards exist that contain principles and terminology, which, under certain circumstances, may be useful in estimating instrument uncertainty. It is acknowledged therefore that concerns exist as to whether the recommended practice is complete in its presentation of acceptable methods. The user is encouraged to review several of the references in the recommended practice that contain other principles and terminology.

The uncertainty and setpoint calculations discussed in this recommended practice may be prepared either manually or with a computer software program. The documentation associated with these calculations is discussed in clause 10; however, the design control and documentation requirements of manual calculations or computer software are outside the scope of the recommended practice.

This recommended practice is intended for use primarily by the owners/operating companies of nuclear power plant facilities or their agents (NSSS, architects, engineers, etc.) in establishing setpoint methodology programs and preparing safety-related instrument setpoint calculations.

This recommended practice utilizes statistical nomenclature that is customary and familiar to personnel responsible for nuclear power plant setpoint calculations and instrument channel uncertainty evaluations. It should be noted that this nomenclature may have different definitions in other statistical applications and is not universal, nor is it intended to be. Furthermore, in keeping with the conservative philosophy employed in power plants calculations, the combination of uncertainty methodology for both dependent and independent uncertainty components is intended to be bounding. That is, the resultant uncertainty should be correct or overly conservative to ensure safe operation. In cases where precise estimation of measurement uncertainty is required, more sophisticated techniques should be employed.

ISA Standard Committee SP67.04 operates as a Subcommittee under SP67, the ISA Nuclear Power Plant Standards Committee, with W. Sotos as Chairman.

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1 Scope

This recommended practice provides guidance for the implementation of ISA-S67.04.01-2000 (equivalent to ANSI/ISA-S67.04, Part I, 1994) in the following areas:

- a) Methodologies, including sample equations to calculate total channel uncertainty
- b) Common assumptions and practices in instrument uncertainty calculations
- c) Equations for estimating uncertainties for commonly used analog and digital modules
- d) Methods to determine the impact of commonly encountered effects on instrument uncertainty
- e) Application of instrument channel uncertainty in setpoint determination
- f) Sources and interpretation of data for uncertainty calculations
- g) Discussion of the interface between setpoint determination and plant operating procedures, calibration procedures, and accident analysis
- h) Documentation requirements

2 Purpose

The purpose of this recommended practice is to present guidelines and examples of methods for the implementation of ISA-S67.04.01-2000 in order to facilitate the performance of instrument uncertainty calculations and setpoint determination for safety-related instrument setpoints in nuclear power plants.

3 Definitions

3.1 allowable value:

a limiting value that the trip setpoint may have when tested periodically, beyond which appropriate action shall be taken.

3.2 analytical limit:

limit of a measured or calculated variable established by the safety analysis to ensure that a safety limit is not exceeded.

3.3 abnormally distributed uncertainty:

a term used in this recommended practice to denote uncertainties that do not have a normal distribution. See 6.2.1.2.2 for further information.

3.4 as found:

the condition in which a channel, or portion of a channel, is found after a period of operations and before recalibration (if necessary).

3.5 as-left:

the condition in which a channel, or portion of a channel, is left after calibration or final setpoint device setpoint verification.

3.6 bias:

an uncertainty component that consistently has the same algebraic sign and is expressed as an estimated limit of error.